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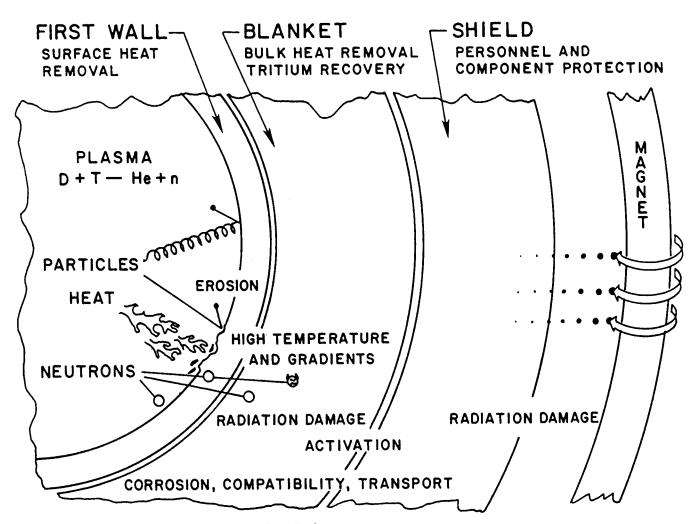
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# FUSION REACTOR ELEMENTS

- PLASMA
- ENGINEERING COMPONENTS
  - MAGNETS
  - PLASMA HEATING
  - NUCLEAR COMPONENTS

#### **NUCLEAR COMPONENTS**

- HANDLING OF PLASMA PARTICLES AND ENERGY CONVERSION
  - PLASMA-INTERACTIVE COMPONENTS (FIRST WALL, Limiter, Divertor, etc.)
  - BLANKET
  - SHIELD
  - TRITIUM SYSTEM



OPERATIONAL CONSIDERATIONS

· EFFICIENT REMOTE MAINTENANCE
FLOW TRANSIENTS
RAPID SHUTDOWN

# WHY SHOULD RESEARCH BE CARRIED OUT <u>NOW</u> ON BLANKET AND MATERIALS ISSUES?

- THE DEVELOPMENT OF A VIABLE FIRST WALL AND BLANKET CONCEPT REPRESENTS A MAJOR, <u>Unresolved Feasibility</u>

  Issue for Fusion.
- THE SELECTION OF A FIRST WALL AND BLANKET CONCEPT CAN SIGNIFICANTLY <u>IMPACT PLASMA ENGINEERING</u> ISSUES, AND VICE VERSA. EXAMPLES INCLUDE:
  - -- PLASMA CONTROL OPTIONS
  - -- IMPURITY CONTROL OPTIONS
  - -- Access and Maintenance
- Operation of Any Fusion Device That Burns Tritium for a Significant Period of Time Will <u>Require</u> Construction of a <u>Tritium-Producing Blanket</u>.
- THE PERCEPTION OF FUSION'S <u>SAFETY AND ENVIRONMENTAL</u>
  FEATURES IS LARGELY DETERMINED BY NUCLEAR/MATERIALS
  TECHNOLOGY CONSIDERATIONS.
- <u>Fusion Economics</u> Will Greatly Depend on the Performance of the Nuclear Systems.
- THE <u>TIME Scale</u> FOR THE DEVELOPMENT OF ADVANCED ALLOYS IS LONG.
- <u>Lessons Learned</u> from Other Technology Development Strongly Suggest Working on Long-Lead Items Early.

#### WHAT WE NEED TO DO

MAINTAIN A <u>Strong</u> and <u>Stable R&D</u> Program Aimed at Resolving the Critical Feasibility Issues of the Fusion Nuclear System:

- -- Perform Separate and Multiple Effects Tests in Non-Fusion Facilities in the 1980's.
- -- PLAN FOR INTEGRAL TESTS IN FUSION FACILITIES IN THE 1990'S.

#### **SUMMARY**

- ONGOING ACTIVITIES IN FUSION NUCLEAR AND MATERIALS
   TECHNOLOGY ARE FOCUSED ON: (1) NEAR-TERM NEEDS FOR
   THE NEXT STEP AND, (2) MAJOR LONGER TERM FEASIBILITY
   Issues.
- Most of the Required Facilities are in Place to Support Near-Term Needs for In-Vessel and First Wall Component Development and Tritium Fuel Handling. There is a Need for a Facility for Large Scale, CW Plasma Simulation Tests of Impurity Control Components.
- THE FOUNDATION OF MATERIALS AND BLANKET DEVELOPMENT
   PROGRAM HAS BEEN ESTABLISHED. RECENT RESULTS ON
   SOLID BREEDER MATERIALS HAVE BEEN QUITE ENCOURAGING.
- THERE IS A NEED FOR ADDITIONAL TEST FACILITIES TO DEVELOP THE MATERIALS DATA BASE AND BLANKET TECHNOLOGY REQUIRED FOR FUSION REACTORS.
- It is Critical for the National Fusion Program to Continue to be <u>Goal-Oriented</u>. This Requires Maintaining an Adequate Program to Demonstrate Engineering Feasibility in the 1990's.

#### FUTURE NUCLEAR SYSTEMS FACILITY REQUIREMENTS

# Non-Fusion Facilities (1980'S)

- Upgrades of Existing Facilities
  - ASURF to 1 MW
  - FELIX TO 4T
  - TSTA/BLANKET PROCESSING
- FIRST WALL/IN-VESSEL COMPONENTS
  - large (~ 1 MW) plasma simulation test facility
  - Combined High Heat Flux and Magnetic Fields for Disruption Simulation
- MATERIALS IRRADIATION FMIT
- Breeder Neutronics Test Facility
  - Neutron Source of  $10^{11} + 10^{12}$  n/sec (use RTNS-II, and/or other facilities)
- LIQUID METAL BREEDER BLANKET TEST FACILITY
  - Combined Testing of Corrosion, MHD, Thermal Hydraulics, and Tritium Recovery

#### FUTURE NUCLEAR SYSTEMS FACILITY REQUIREMENTS (CONT.)

- SOLID BREEDER BLANKET TEST FACILITY
  - <u>Combined</u> Testing of Thermal-Hydraulic/Thermal-Mechanical, Tritium Release & Recovery
    - Non-Nuclear Simulation of Bulk Heating
    - Fission Reactors

#### MAINTENANCE

- Mock-up
- COMPUTER GRAPHIC SIMULATION STUDIES

# FUSION FACILITIES (1990'S)

- Fusion Based Large Scale First Wall/Blanket/Shield
   Component and Materials Testing Facility
  - Large Test Volume/Area (> 10 m<sup>2</sup>)
  - NEUTRON WALL LOADING ~ 2 MW/m<sup>2</sup>
  - HIGH AVAILABILITY

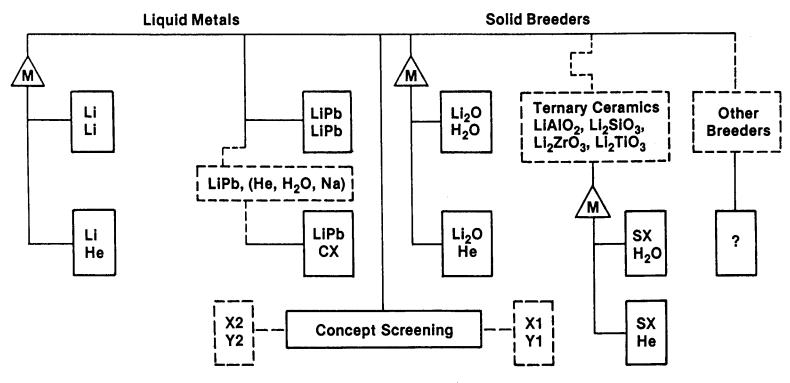
# KEY FEASIBILITY ISSUES FOR IN-VESSEL AND FIRST WALL COMPONENTS

	ISSUE		ACTIVITY/FACILITY			
•	Impurity Control					
		PLASMA CONTAMINATION HIGH HEAT FLUXES PARTICLE PUMPING	PMTF, PISCES, TEXTOR LIMITER ASURF/PMTF PISCES			
•	Сомр	ONENT LIFETIME				
		Erosion/Redeposition Disruptions Electrogmagnetic Loads Corrosion/Compatibility Radiation Effects	PMTF, PISCES ESURF, ASURF FELIX VARIOUS LOOPS DUAL-ION IRRADIATIONS, FISSION REACTORS RTNS-II (FMIT)			
•	TRIT	IUM CONTAINMENT	SMALL LAB. EXPERIMENTS			
		_	10495			

ENERGY RECOVERY

**ASURF** 

#### **BLANKET OPTIONS**



A concept for the detailed comparative study.

Structural Material

- Big difference in R&D
- Crucial for concept selection
  - (1) PCA
  - (2) Advanced alloy (FS or V or ?)

 A study required to define a concept (within a class) for inclusion in the detailed comparative study.

#### M = Neutron Multiplier

- All breeders (except LiPb may require multiplier.
- Is beryllium the only choice?
- Beryllium assessment.

#### KEY FEASIBILITY ISSUES FOR BLANKETS - GENERAL

#### ISSUE

### ACTIVITY/FACILITY

• STRUCTURAL LIFETIME

- DUAL-ION IRRADIATIONS
- FISSION REACTORS (ORR,
  - HFIR)
- RTNS-II
  - (FMIT)

- ELECTROMAGNETIC EFFECTS
   (FIELD PENETRATION/
   DISTURBANCES)
- FELIX

Assembly/Maintenance

- TFTR
- SMALL SCALE TESTS
- ACTIVATION, WASTE
   MANAGEMENT, REPROCESSING
- PANEL REPORT ON LOW ACTIVATION
- SMALL SCALE STUDIES
- EFFICIENT HEAT EXTRACTION

# ALLOY DEVELOPMENT FOR IRRADIATION PERFORMANCE

# • NEAR TERM

PATH A AUSTENITIC STAINLESS STEEL

PATH B NICKEL-BASE ALLOYS (DROPPED)

PATH E FERRITIC STEELS

# Long Term

PATH C REACTIVE/REFRACTORY ALLOYS

PATH D INNOVATIVE CONCEPTS

#### KEY FEASIBILITY ISSUES FOR SOLID BREEDER BLANKETS

#### ISSUE

#### ACTIVITY/FACILITY

• TRITIUM BREEDING

- Cooperative Integral Breeder Neutronics Experiments at JAERI

- TRITIUM RELEASE
  - -- IN SITU RECOVERY
  - -- MATERIALS DATA

- TRIO/ORR
- FUBR/EBR-II, LAB-EXPERIMENTS

THERMAL PERFORMANCE

- Small Scale Heat Transfer and He Purge Flow Tests

- Lifetime
  - -- NEUTRON EFFECTS
  - -- CORROSION

- FUBR/EBR-II
- Small Scale Lab. Experiments

#### KEY FEASIBILITY ISSUES FOR LIQUID BREEDER BLANKETS

#### ISSUE

# ACTIVITY/FACILITY

- Corrosion/Compatibility
- LOOP EXPERIMENTS

MHD

- Program Starting in FY 1984

TRITIUM RECOVERY

- No Work at Present (Past Experiments with Li)

#### KEY FEASIBILITY ISSUES FOR SHIELDING

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#### ACTIVITY/FACILITY

Predict Radiation
 Attenuation

- INTEGRAL SHIELDING FACILITY
- Define Radiation Protection Requirements for Reactor Components, Access
- IPNS, OTHER FACILITIES RTNS-II

• ELECTROMAGNETICS

FELIX

• Cost

DESIGN STUDIES

# KEY FEASIBILITY ISSUES FOR TRITIUM FUEL CYCLE

### ISSUE

ACTIVITY/FACILITY

**TSTA** 

- TRITIUM REPROCESSING FROM VACUUM STREAM
- TRITIUM ACCOUNTABILITY
- TRITIUM CONTAINMENT
- TRITIUM REPROCESSING FROM FUTURE UPGRADES OF TSTA BLANKET